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Title: Characterization of Delayed-Neutron and Delayed-Gamma Pyroprocessing

Emission Signatures Using MCNP6

Author(s): Durkee, Joe W. Jr.

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Characterization of Delayed-Neutron and Delayed-Gamma Pyroprocessing Emission Signatures Using MCNP6 Joe W. Durkee, Jr. October, 2012

A computational non-destructive safeguards study has been conducted using the MCNP6¹ radiation-transport code to examine delayed-neutron (DN) current and delayed-gamma (DG) activity ratio (AR) signatures for a conceptual electrorefining and pyroprocessing (EP) model. Spent-fuel (SNF) inventories were prepared using MCNP6 burnup calculations for a generic Westinghouse fuel assembly² with 3% and 5% ²³⁵U enrichments; 20-, 30-, 40-, and 50-GWd/MTU burnups; and 3-, 5-, 10-, 20-, and 30-year cooling times to facilitate parametric evaluations. The electrorefining model was patterned after the INL Mark-IV electrorefiner (ER),³ and treats the distribution of active metals, noble metals, lanthanides, uranium, and TRUs following fuel dissolution and transport to the molten salt eutectic, liquid cadmium cathode (LCC), and cathode mandrel. Signatures have been assessed before and after electrorefining. DG ARs for ¹³⁴Cs/¹³⁷Cs, ¹³⁴Cs/¹⁵⁴Eu, and ¹⁵⁴Eu/¹³⁷Cs were developed, and preliminary assessment of new DG ARs that may be better suited for pyroprocessing safeguards was done. Figure 1 shows calculated DN and DG fluxes for 3% enrichment, 20 GWd/MTU burnup, 3 year cooling before electrorefining, with SNF in the ER, and after electrorefining, with material assignment to processing units. Our work suggests that DN and DG markers can be useful for EP forensic analysis ____

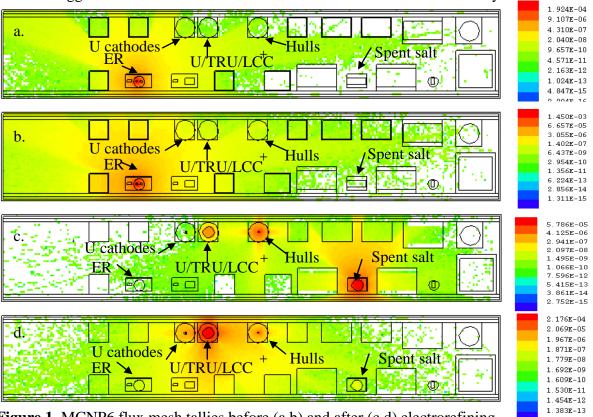


Figure 1. MCNP6 flux mesh tallies before (a,b) and after (c,d) electrorefining. a,c: Photon (photons/cm²-s). b,d: Neutron (neutrons/cm²-s).

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